

UNITED STATES

NUCLEAR REGULATORY COMMISSION

REGION II

230 PEACHTREE STREET, N.W. SUITE 1217

ATLANTA, GEORGIA 30303

NOV 2 2 1977

Contractic

In Reply Refer To:

RII:JPO

50-438, 50-439

50-259, 50-260

50-296, 50-518

50-290, 50-510

50-519, 50-520

50-521, 50-553

50-554, 50-327 50-328, 50-390

50-391, 50-566

50-567

Tennessee Valley Authority
Attn: Mr. Godwin Williams, Jr.
Manager of Power
830 Power Building
Chattanooga, Tennessee 37401

Gentlemen:

Enclosed is IE Bulletin No. 77-06 which requires action by you with regard to your power reactor facility(ies) with an operating license.

Should you have questions regarding this Bulletin or the actions required of you, please contact this office.

Sincerely,

James P. O'Reilly

Director

Enclosures:

1. IE Bulletin No. 77-06

2. List of IE Bulletins
Issued in 1977

cc: J. E. Gilleland
Assistant Manager of Power
830 Power Building
Chattanooga, Tennessee 37401

W. W. Aydelott, Project Manager Bellefonte Nuclear Plant P. O. Box 2000 Hollywood, Alabama 35752 -2-

(cc's continued)

. ;

Stan Duhan 400 Commerce Street E4D112 Knoxville, Tennessee 37902

J. G. Dewease, Plant Superintendent Box 2000 Decatur, Alabama 35602

R. T. Hathcote, Project Manager Hartsville Nuclear Plant P. O. Box 2000 Hartsville, Tennessee 37074

G. G. Stack, Project Manager Sequoyah Nuclear Plant P. O. Box 2000 Daisy, Tennessee 37319

J. M. Ballentine Plant Superintendent Sequoyah Nuclear Plant P. O. Box 2000 Daisy, Tennessee 37319

T. B. Northern, Jr.
Project Manager
Watts Bar Nuclear Plant
P. O. Box 2000
Spring City, Tennessee 37381

Address list of utilities receiving Bulletin 77-06.

Office of Inspection and Enforcement Bulletin 77-06

9250 West Flagler Street Miami, Florida 33101

Addresses		In Reference To:	
1.	Alabama Power Company ATTN: Mr. A. R. Barton Senior Vice President 600 N. 18th Street Post Office Box 2641 Birmingham, Alabama 35291	50-348 Farley Unit 1 50-364 Farley Unit 2	
2.	Allied-General Nuclear Services ATTN: Mr. Raymond C. Baxter President P. O. Box 847 Barnwell, South Carolina 29812	50-332 Barnwell Nuclear Fuel Plant	
3.	Carolina Power and Light Company ATTN: Mr. J. A. Jones Executive Vice President Engineering, Construction and Operation 336 Fayetteville Street Raleigh, North Carolina 27602	50-325 Brunswick Unit 1 50-324 Brunswick Unit 2 50-400 Harris Unit 1 50-401 Harris Unit 2 50-402 Harris Unit 3 50-403 Harris Unit 4 50-261 Robinson Unit 2	
4.	Duke Power Company ATTN: Mr. W. H. Owen, Vice President Engineering 422 South Church Street P. O. Box 2178 Charlotte, North Carolina 28242	50-413 Catawba Unit 1 50-414 Catawba Unit 2 50-491 Cherokee Unit 1 50-492 Cherokee Unit 2 50-493 Cherokee Unit 3 50-488 Perkins Unit 1 50-489 Perkins Unit 2 50-490 Perkins Unit 3	
5.	Duke Power Company ATTN: Mr. William O. Parker, Jr. Vice President of Steam Production 422 South Church Street P. O. Box 2178 Charlotte, North Carolina 28242	50-369 McGuire Unit 1 50-370 McGuire Unit 2 50-269 Oconee Unit 1 50-270 Oconee Unit 2 50-287 Oconee Unit 3	
6.	Florida Power and Light Company ATTN: Dr. R. E. Uhrig, Vice President of Nuclear and General Engineering P. O. Box 013100	50-335 St. Lucie Unit 1 50-389 St. Lucie Unit 2 50-250 Turkey Point Unit 3 50-251 Turkey Point Unit 4	

Addresses	In Reference To:
7. Florida Power Corporation ATTN: Mr. W. T. Stewart Director of Power Productio P. O. Box 14042, Mail Stop C-4 St. Petersburg, Florida 33733	50-302 Crystal River Unit 3
8. Georgia Power Company ATTN: Mr. J. H. Miller, Jr. Executive Vice President 270 Peachtree Street, N. W. Atlanta, Georgia 30303	50-321 Hatch Unit 1 50-366 Hatch Unit 2 50-424 Vogtle Unit 1 50-425 Vogtle Unit 2
9. Mississippi Power and Light Compan ATTN: Mr. Norris L. Stampley Vice President of Productio P. O. Box 1640 Jackson, Mississippi 39205	50-417 Grand Gulf Unit 2
10. Offshore Power Systems ATTN: Dr. D. H. Walker, Manager 8000 Arlington Expressway P. O. Box 8000 Jacksonville, Florida 32211	50-437 FNP 1-8
11. South Carolina Electric and Gas Con ATTN: Mr. M. C. Johnson, Vice Pres Special Services P. O. Box 764 Columbia, South Carolina 29202	
50-52 50-55 50-32 50-32 50-39 50-39	50-438 Bellefonte Unit 1 50-439 Bellefonte Unit 2 50-259 Browns Ferry Unit 1 50-260 Browns Ferry Unit 2 50-296 Browns Ferry Unit 3 50-518 Hartsville Unit 1 50-519 Hartsville Unit 2 20 Hartsville Unit 3 21 Hartsville Unit 4 53 Phipps Bend Unit 1 50-554 Phipps Bend Unit 2 27 Sequoyah Unit 1 28 Sequoyah Unit 1 29 Watts Bar Unit 2 30 Yellow Creek Unit 1

Addressee

13. Virginia Electric and Power Company
ATTN: Mr. W. L. Proffitt
Senior Vice President
P. O. Box 26666
Richmond, Virginia 23261

In Reference To:

50-338 North Anna Unit 1 50-339 North Anna Unit 2 50-404 North Anna Unit 3 50-405 North Anna Unit 4 50-280 Surry Unit 1 50-281 Surry Unit 2

NUCLEAR REGULATORY COMMISSION OFFICE FOR INSPECTION AND ENFORCMENT WASHINGTON, D. C. 20555

November 22, 1977

IE Bulletin No. 77-06

POTENTIAL PROBLEMS WITH CONTAINMENT ELECTRICAL PENETRATION ASSEMBLIES

Description of Circumstances:

On October 3, 1977, Northeast Nuclear Energy Company reported to the NRC Region I Office that two control valves installed inside containment at Millstone Unit No. 2 demonstrated abnormal operational characteristics. The licensee reported that an unexpected closure of a letdown flow stop valve occurred. While investigating this problem, the normally closed safety injection recirculation return line drain valve was found to be in the open position. Investigation of these events revealed the cause for failure to be electrical shorts between conductors within a containment low voltage penetration assembly.

The licensee subsequently determined that the wiring for both of the valves shared the same low voltage module in an electrical penetration. Electrical tests by the licensee revealed that 15 of the 85 conductors in the suspect connector module exhibited decreased insulation resistance between conductors. Based on this finding, it is believed that an electrical path between adjacent circuits in the connector module was established. This resulted in spurious operation of the valves. Similar resistance checks performed on the remaining low voltage modules within the affected penetration assembly revealed 17 additional conductors with reduced insulation resistances. All conductors with resistances less than 20 megolms were disconnected and their circuits were reconnected through spare conductors.

Examination of the three remaining low voltage penetration assemblies, identified 7 additional conductors with resistances of less than 20 megohms. Each of these circuits were also reconnected through a spare conductor.

November 22, 1977

Investigation showed that the reduced insulation resistance was probably caused by moisture accumulation within the penetration assembly together with small fissures in the epoxy seals surrounding each conductor in the module. The licensee believes that moisture penetrating these cracks reduced the insulation resistance between adjacent conductors. To prevent further degradation from moisture buildup within the penetration assemblies, the licensee re-established a dry nitrogen pressure of 24 PSIG in the penetrations.

Subsequently the licensee reported that a second event of a similar nature occurred on October 14, 1977. In this instance the sample isolation valve for the pressurizer surge line failed to close on command. Investigation into this event indicated that electrical shorts between conductors due to a moisture accumulation problem was the probable cause for valve misoperation. The shorted wires were disconnected and the valve was de-energized in the closed position.

In discussions on the issue with the licensee and the electrical penetration vendor, General Electric Company, NRC staff determined that maintenance of nitrogen pressure is essential to the integrity of both high and low voltage penetration assemblies. The General Electric Company specifies in its penetration assembly maintenance and operation manual that a 15 PSIG dry nitrogen pressure should be maintained on low voltage units while 30 PSIG should be maintained on high voltage units.

Action To Be Taken By Licensees Of All Power Reactor Facilities With An Operating License:

Containment Electrical Penetrations - For safety related systems

- 1.0 Do you have containment electrical penetrations that are of the G. E. Series 100, or are otherwise similar in that they depend upon an epoxy sealant and a dry nitrogen pressure environment to ensure that the electrical and pressure characteristics are maintained so as to ensure the functional capability as required by the plant's safety analysis report; namely, (1) to ensure adequate functioning of electrical safety-related equipment and (2) to ensure containment leak tightness?
- 1.1 Have you experienced any electrical failures with this type of penetration?

- 2.0 For those penetrations referenced in Item 1 above, have you maintained the manufacturer's prescribed nitrogen pressure at all times?
- 2.1 If you have operated the penetrations without maintaining a nitrogen pressure was any degradation of insulation resistance or anomolous component operation detected?
- 2.2 If no measurements were taken during periods when nitrogen pressure was not maintained, how were you assured that the insulation resistance was not degrading or degraded?
- 2.3 How do you determine that circuit insulation resistances values are satisfactorily maintained?
- 3.0 Is there a need, as determined by either the vendor or yourself, to maintain penetrations pressurized during a LOCA?
- 3.1 What measures have you taken to ensure that penetrations of this type will perform their design function under LOCA conditions? (design reviews, analyses or tests)
- 3.2 Are the measures that provide this assurance adequate to satisfy the Commission's regulations (GDC 4, Appendix A to Part 50; QA Criteria, Appendix B to Part 50)
- 4.0 Provide your response to Items 1.0 through 3.2 above in writing within 10 days. In addition, provide an oral response by 4:00 p.m. (Local Time) November 25, 1977. Responses should be submitted to the Director of the appropriate NRC Regional Office. A copy of written responses should be forwarded to the U.S. Nuclear Regulatory Commission, Office of Inspection and Enforcement, Division of Reactor Operations Inspection, Washington, D. C. 20555.

Approved by GAO, B180225 (R0072); clearance expires 7-31-80. Approval was given under a blanket clearance specifically for identified generic problems.

IE Bulletin 77-06 November 22, 1977

LISTING OF IE BULLETINS ISSUED IN 1977

Bulletin No.	Subject	Date Issued	Issued To
77-05A	Supplement 77-05A to IE Bulletin No. 77-05 - Electrical Connector Assemblies	11/15/77	All PWR Power Reactor Facilities with an Operating License (OL) or Construction Permit (CP)
77-05	Electrical Connector Assemblies	11/8/77	All PWR Power Reactor Facilities with an Operating License (OL) or Construction Permit (CP)
77-04	Calculational Error Affecting the Design Performance of a System for Controlling pH of Containment Sump Water Following a LOCA	11/4/77	All PWR Power Reactor Facilities with an Operating License (OL) or Construction Permit (CP)
77-03	On-Line Testing of the W Solid State Protection System	9/12/77	All W Power Reactor Facilities with an Operating License (OL) or Construction Permit (CP)
77-02	Potential Failure Mechanism in Certain W AR Relays with Relays with Latch Attachments	9/12/77	All Holders of Operating Licenses (OL) or Construction Permits (CP)
77-01	Pneumatic Time Delay Set Point Drift	4/29/77	All Holders of Operating Licenses (OL) or Construction Permits (CP)